

An efficient numerical tool for dose deposition prediction applied to synchrotron medical imaging and radiation therapy

Alberto Mittone,^{a,b,*} Fabien Baldacci,^c Alberto Bravin,^d Emmanuel Brun,^{a,d} François Delaire,^c Claudio Ferrero,^d Sergei Gasilov,^a Nicolas Freud,^c Jean Michel Létang,^c David Sarrut,^c François Smekens^c and Paola Coan^{a,b}

^aDepartment of Physics, Ludwig Maximilians University, Am Coulombwall 1, Munich, Germany,

^bDepartment of Clinical Radiology, Ludwig Maximilians University, Munich, Germany, ^cUniversité de Lyon, CREATIS; CNRS UMR5220; Inserm U1044; INSA-Lyon; Université Lyon 1; Centre Léon Bérard, France, and ^dEuropean Synchrotron Radiation Facility, Grenoble, France.

E-mail: alberto.mittone@physik.uni-muenchen.de

Medical imaging and radiation therapy are widely used synchrotron-based techniques which have one thing in common: a significant dose delivery to typically biological samples. Among the ways to provide the experimenters with image guidance techniques indicating optimization strategies, Monte Carlo simulation has become the gold standard for accurately predicting radiation dose levels under specific irradiation conditions. A highly important hampering factor of this method is, however, its slow statistical convergence. A track length estimator (TLE) module has been coded and implemented for the first time in the open-source Monte Carlo code *GATE/Geant4*. Results obtained with the module and the procedures used to validate them are presented. A database of energy-absorption coefficients was also generated, which is used by the TLE calculations and is now also included in *GATE/Geant4*. The validation was carried out by comparing the TLE-simulated doses with experimental data in a synchrotron radiation computed tomography experiment. The TLE technique shows good agreement *versus* both experimental measurements and the results of a classical Monte Carlo simulation. Compared with the latter, it is possible to reach a pre-defined statistical uncertainty in about two to three orders of magnitude less time for complex geometries without loss of accuracy.

Keywords: Monte Carlo; X-rays; fast dose simulation; *Geant4/GATE*.

1. Introduction

Knowledge of the dose delivered during medical imaging and radiation therapy with a synchrotron source represents a considerable issue. In order to preserve the integrity of biological samples (but also other samples), their exposure should be carefully guided following dosimetric criteria which all lead to the basic paradigm that the absorbed dose should be as low as reasonably achievable. In this respect, therapeutic and imaging doses, when combined, cannot be regarded from the same viewpoint because the imaging dose adds to an already high level of therapeutic radiation.

The Monte Carlo (MC) method is often used to compute the distribution of the dose deposited in both imaging and radiotherapy applications. It provides accurate results but still requires long computation times, hindering its use in clinical routine applications. Many MC codes are available to simulate radiation transport (see Table 1).

Table 1

Overview of the main MC codes to simulate radiation transport.

MC code	Reference
<i>Geant4</i>	Agostinelli <i>et al.</i> (2003), Allison <i>et al.</i> (2006)
<i>MCNPX</i>	Pelowitz (2005)
<i>FLUKA</i>	Battistoni <i>et al.</i> (2007), Ferrari <i>et al.</i> (2005)
<i>EGSnrc</i>	Fragoso <i>et al.</i> (2009)
<i>PENELOPE</i>	Salvat & Fernández-Varea (2009), Salvat <i>et al.</i> (2011)

They are commonly used for research purposes, either in simulation mode or in combination with various acceleration techniques, such as condensed history methods or variance reduction techniques (VRT), *e.g.* splitting and Russian roulette, importance sampling, forced detection, *etc.* (Jenkins *et al.*, 1988). These acceleration techniques provide substantial speed-up but have to be used very cautiously to stay within their limits of validity. Among the VRTs, the track length estimator (TLE) method is known as a very efficient tallying

method, well suited for kerma calculations at any given point during photon irradiations (Williamson, 1987).

We investigate the application of TLE to speed up dose calculations for low-energy (from 1 keV to a few hundred keV) photon irradiations, which include radiology, brachytherapy, synchrotron radiation therapy and imaging applications. This article is structured as follows: in §2, the TLE method and the related physical issues are first introduced, its implementation in the next release of the open source code *GATE* (*Geant4 Application for Emission Tomography*, version 9.5) (Jan *et al.*, 2004, 2011) is described, then the test cases for validation are presented as well as the dose measurement protocol and associated set-up; §3 presents the results of the TLE implementation against analogous MC simulations and then their validation against experimental data obtained with a high-resolution computed tomography (CT) test case using monochromatic X-rays (issued from a synchrotron radiation source). Finally, the TLE method is shown to properly cope with very complex geometries with no need for heavy computing resources.

2. Materials and methods

2.1. The TLE method

The use of a TLE technique for calculating particle fluences, kerma and absorbed doses has long been known (Carlsson, 1985; Williamson, 1987). It is implemented, for example, in the *MCNPX* MC code (Demarco *et al.*, 2002; Smans *et al.*, 2010) and in some dedicated tools, notably for external radiotherapy (van der Zee *et al.*, 2005) and brachytherapy applications (Chibani & Williamson, 2005; Taylor *et al.*, 2007). Considering a monoenergetic photon beam with energy E , the absorbed dose in charged particle equilibrium (CPE) is given by (Carlsson, 1985; Berger *et al.*, 2010)

$$D = \Phi E \mu_{\text{en}}/\rho, \quad (1)$$

where Φ is the particle fluence and μ_{en}/ρ is the mass energy-absorption coefficient. Fluence can be interpreted as the track length density of the particles at a point \mathbf{r} in space (Carlsson, 1985), *i.e.*

$$\Phi(\mathbf{r}) = dL(\mathbf{r})/dV. \quad (2)$$

For a photon traversing a voxel of volume V , an estimate of the corresponding fluence is therefore given by (Williamson, 1987; Demarco *et al.*, 2002)

$$\Phi = L/V, \quad (3)$$

where L is the track length, *i.e.* the straight-line distance travelled in the voxel between successive collisions. TLE can thus be used to determine the expected value of the dose deposited along every photon trajectory given by (1). With the TLE scoring method, a photon deposits its energy in all voxels it encounters between successive interaction points, instead of doing so only at interaction points as is the case in an equivalent MC simulation. The major advantage of the use of the TLE is a drastic acceleration of the convergence of the simulation.

2.2. Local energy deposition by secondary electrons

According to the TLE method, secondary electrons are not tracked and their energy is deposited locally (within a single voxel). This approximation is satisfactory when the CPE condition is fulfilled and the electron range remains smaller than either the voxel size or the required spatial accuracy. Comparing the electron range in the continuous-slowing-down approximation (CSDA range) with the voxel size is a conservative criterion, since the projected range is always smaller than the CSDA range (detour factor below 1) (Berger *et al.*, 2005). For soft tissues (mass density 1.06 g cm^{-3}), the CSDA range stays below 1 mm up to about 300 keV (see ICRU, 1984).

2.3. Energy-absorption tables

In the energy domain of interest in this study (keV X-rays), the linear energy-absorption (μ_{en}) and energy-transfer (μ_{tr}) coefficients can be considered to be the same quantity (Attix, 2004; Berger *et al.*, 2010; Freud *et al.*, 2008),

$$\mu_{\text{en}} \simeq \mu_{\text{tr}}. \quad (4)$$

For tissues composed of elements with atomic numbers $Z \leq 20$, the relative difference between μ_{tr} and μ_{en} remains below 1% for energy values up to 1 MeV. The difference can reach a few percent at 1 MeV for elements with $Z > 20$ (Attix, 2004). In the case of a mixture of elements, the mass energy-transfer coefficient μ_{tr}/ρ satisfies the additivity rule.

The values of μ_{en}/ρ may be obtained in several ways:

(i) Directly from the NIST database for elemental media and a limited number of materials of interest (Berger *et al.*, 2010). In the case of compounds or mixtures, the additivity rule can be used as an approximation.

(ii) Using the EPDL97 database and the approximation by equation (4). Details about the calculation of the coefficients can be found on the NIST webpage (Berger *et al.*, 2010).

(iii) Using MC simulations [method described by Freud *et al.* (2008)]. In *Geant4*, several low-energy electromagnetics physics lists based on Livermore or *PENELOPE* models (Ivanchenko *et al.*, 2011) are available for reference. Both use EPDL97 and EADL to describe photon interactions and atomic relaxation, respectively. As regards Compton scattering, both Livermore and *PENELOPE* models predict the ionized shell and subsequent atomic relaxation. Note, however, that the fluorescence emission following Compton scattering is a very rare process and has a negligible influence on the μ_{en} value.

Fig. 1 presents a comparison between μ_{tr}/ρ calculated for cortical bone and lung tissue (ICRU, 1989) using the EPDL97 database (Cullen *et al.*, 1997), see (ii), μ_{tr}/ρ derived from *Geant4* (version 9.5) simulations (iii) with the Livermore model, and μ_{en}/ρ from NIST (Berger *et al.*, 2010), (i). No significant difference is detected between these three sets of data.

In the case of iodine (Fig. 2), slight differences are found between the μ_{tr}/ρ calculated *via* EPDL97 and μ_{en}/ρ from NIST, up to about 4% at 1 MeV, corresponding to the average

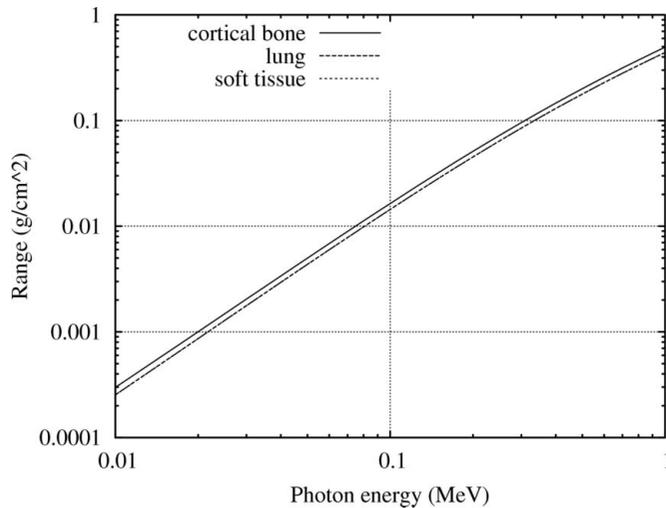


Figure 1
Comparison between energy absorption-to-attenuation coefficient ratios calculated for cortical bone and lung tissue (ICRU, 1989) using the EPDL97 database (Cullen *et al.*, 1997), as derived from *Geant4* (version 9.5) simulations with the Livermore model [according to the method described by Freud *et al.* (2008)], and as given by NIST (Berger *et al.*, 2010).

fraction of the kinetic energy of secondary charged particles escaping through radiative processes (Berger *et al.*, 2010). Moreover, no significant difference (below about 1%) appears between the μ_{tr}/ρ values calculated through EPDL97 and estimated by *Geant4*, whichever model (*PENELOPE* or Livermore) is used.

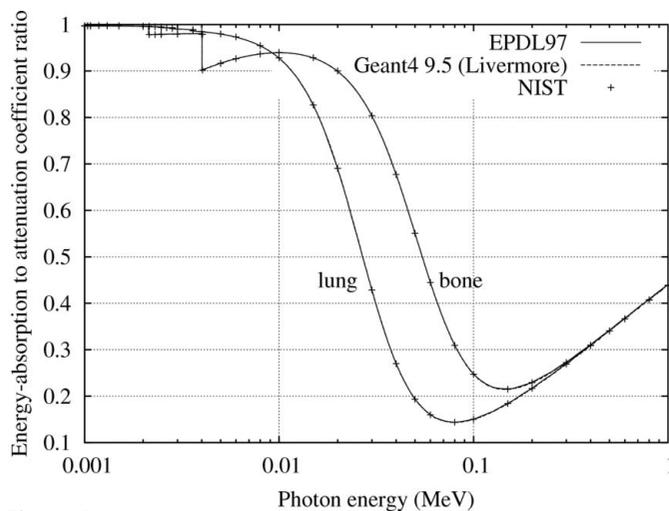


Figure 2
Comparison between energy absorption-to-attenuation coefficient ratios calculated for iodine using the EPDL97 database (Cullen *et al.*, 1997), simulated using *Geant4* (version 9.5) in conjunction with the *PENELOPE* and Livermore physics models, and as provided by NIST (Berger *et al.*, 2010). The curves corresponding to the *Geant4*, *PENELOPE* and Livermore models can barely be distinguished. The relative difference between EPDL97 and *Geant4* stays below about 1%. When energy increases, the difference between μ_{tr}/ρ (either from EPDL97 or *Geant4*) and μ_{en}/ρ from NIST increases up to about 4% at 1 MeV. In the EPDL97 library the Doppler broadening effect is not considered. It has been shown (Ye *et al.*, 2006) that the usage of this approximation compared with a more advanced model (impulse approximation) does not produce significant differences in the results, at least for the considered energy range.

The database we generated from the EPDL97 library, included in the next release of *GATE*, incorporates the μ_{en} coefficients mentioned above for all elements with $1 \leq Z \leq 100$ in the energy range [1 keV, 1 MeV] and has been used in all TLE calculations reported here.

2.4. Implementation of the TLE in *GATE*

During the initialization of a simulation, the pre-generated database, presented in §2.3, is loaded. The coefficients are tabulated for a set of 117–868 energies and for each Z value.

The discontinuities are treated by insertion of two identical energies with different μ_{en} values. When the energy of the photon is equivalent to the absorption edge energy the higher values of μ_{en} are considered for the simulation.

Materials are defined as mixtures of n elements with corresponding mass fractions w_n . When an energy value used by the simulation does not appear in the database, the value of the corresponding μ_{en} is calculated on the fly by means of a logarithmic interpolation between the μ_{en} values referring to the two nearest energies. We implemented this method in *GATE* (version 6.2) and it will be available for use in the next release of *GATE*. In order to use it, it is sufficient to call *TLEDoseActor* in a way similar to the pre-existing *DoseActor* (Jan *et al.*, 2011). The distribution maps of the energy deposited in the volume V of interest are created if the actor is attached to V . The volume must be a matrix of voxels named *NestedParametrizedVolume*. By definition, one element of the dose score matrix corresponds to one *doseI*. The implementation of a look-up table to store the tabulated values μ_{en} of the already defined materials is foreseen to accelerate the computations.

2.5. Benchmarking of the TLE implementation in *GATE* against MC simulation

The test case chosen for the comparison between TLE and equivalent classical MC simulations is a chest radiography consisting of a CT model (obtained during a real CT acquisition and provided in the set of examples distributed with *GATE*) of a thorax phantom (512 × 512 × 53 voxels of 0.602 mm × 0.602 mm × 3.000 mm; see Fig. 3) irradiated by an

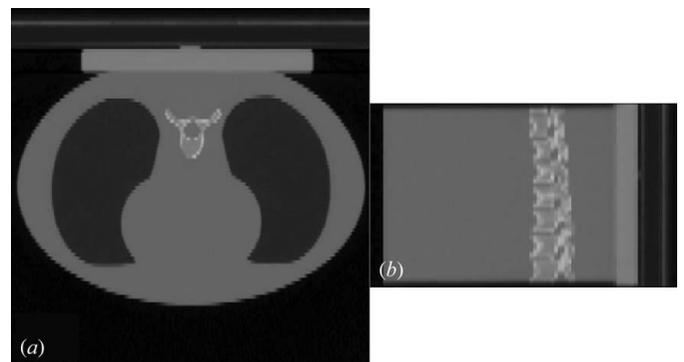


Figure 3
CT model of the thorax phantom used in the chest radiography simulation test case. (a) Transverse slice; (b) sagittal slice.

X-ray tube, simulated as a point source with a 81 kVp polychromatic spectrum. The thorax phantom is the Dynamic Thorax Phantom from CIRS Inc. It is composed of several different materials (in terms of both density and linear attenuation) included between air and metal implant. The distance from the source to the phantom centre was set to 100 cm. Simulations were carried out using *Geant4* (version 9.5) with the *PENELOPE* physics models. The voxel size of dose and energy deposited maps was set to 3.08 mm × 3.08 mm × 9.9375 mm.

2.6. Experimental validation of the TLE

Measurements were performed at the biomedical beamline (ID17) of the European Synchrotron Radiation Facility (ESRF, Grenoble, France), pursuing a twofold objective: (i) evaluating the photon flux, *i.e.* the photon fluence per unit time and per storage-ring electron current necessary to estimate the number of histories to be used in the simulations, and (ii) determining dose values in different experimental configurations to be compared with dosimetric results simulated by our TLE code.

2.6.1. Photon flux measurement protocol. In the simulations, the number of histories n_h needed to compute the total deposited dose is given by $n_h = \varphi ItS$, where S is the beam cross-sectional surface area, I is the storage ring electron current, t is the exposure time and φ is the photon flux (photons $\text{s}^{-1} \text{mm}^{-2} \text{mA}^{-1}$).

The photon flux was derived from the dose absorbed in water dosimetry measurements, carried out using a ionization chamber connected to a PTW dosimeter (PTW 31010 calibrated by PTB Freiburg, Germany) at different X-ray energies in the range 33.7–72 keV. The active volume of the ionization chamber is 0.125 mm³. As the height of the synchrotron beam is smaller than the chamber size, measurements were performed by vertically scanning the ionization chamber with the photon beam (Prezado *et al.*, 2011). The chamber was positioned perpendicularly to the direction of the beam: in this case no correction concerning the directional dependence of the device had to be applied. A correction taking into account the energy behaviour of the chamber was applied. The correction factor curves are provided directly by the manufacturer of the chamber. The measurements were repeated twice and the mean value was taken as the reference estimate. The photon flux at the point of measurement was evaluated as

$$\varphi_{\text{PTW}} = \frac{D\rho}{E\mu_{\text{en}}tI}, \quad (5)$$

where D is the dose recorded by the ionization chamber. Note that equation (5) derives from equation (1) (keeping the same notations), and is valid in the case of a monochromatic X-ray beam with negligible scattering, which is a satisfactory approximation when the ionization chamber is placed in air.

A high-efficiency germanium detector (HEGD) was used to cross-check the results delivered by the PTW dosimeter. The HEGD consists of a high-purity germanium detector operating at liquid-nitrogen temperature. It is segmented into two

rows of 10 mm height and 432 parallel strips with 0.35 mm pitch. The number of counts was determined using the detector-specific energy-dependent photon-counts relation, averaging over repeated acquisitions. From the mean value b of counts (expressed in bits), the photon flux at the point of measurement was calculated as

$$\varphi_{\text{HEGD}} = kb/S_{\text{pix}}tI, \quad (6)$$

where k is the bit–photon conversion factor (corresponding to the number of photons integrated by the HEGD per bit) and S_{pix} is the irradiated pixel surface area (Coan *et al.*, 2006).

2.6.2. Dose measurement protocol. Measurements of the dose deposited at different positions within a cylindrical plastic phantom were performed using a CT ionization chamber (Radcal 10X6-3CT, Monrovia, USA) with a read-out connection to a Radcal dosimeter. The Radcal chamber is designed for CT dose measurements and provides dose values in water. The phantom consists of an external layer of polymethylmethacrylate (PMMA) with an outer diameter $D = 100$ mm and height of 150 mm, and an internal cylinder of polyethylene (PE) of 60 mm diameter (Fig. 4). Three holes, 13 mm in diameter, make it possible to insert the ionization chamber at different positions (0, 18, 40 mm from the centre). The experiment was carried out with a monochromatic X-ray beam of 60 keV. The phantom was rotated by 360° around its central axis (perpendicular to the beam direction, see arrow in Fig. 4) at a speed of 5° s⁻¹. The X-ray beam used had a parallelepipedal shape with a cross section equal to 2 mm (V) × 100 mm (H). The holes were filled by cylinders of PE, except for the one containing the chamber.

2.6.3. Simulation parameters. Simulations were carried out using *Geant4* (version 9.5) with the *PENELOPE* physics models. An *ad hoc* electron cut-off value of 100 keV was used in order to avoid tracking any secondary electron. The distance between the source and the sample was set to 50 cm. The measurements performed with the Radcal chamber on the CT cylindrical phantom were simulated with the active volume of the ionization chamber, 3 cm³, being replaced by a water volume (the chamber provides dose-to-water values). For the simulation, a voxel size of 0.2 mm × 0.2 mm × 1 mm was chosen and the dose estimates were computed by averaging the values obtained in all the voxels contained in the volume of the chamber’s active region. The number of histories chosen for the simulations was 10⁹ in order to obtain

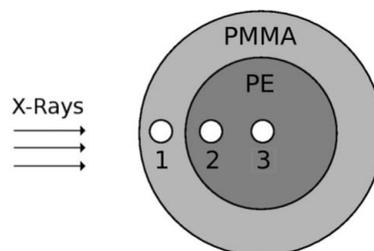


Figure 4 Phantom geometry and experimental configuration used in the CT measurements.

a mean statistical uncertainty below 1% in the central irradiated slice

2.7. Performances comparison between TLE and MC dose deposition simulations in anatomically complex cases

In order to assess the potential of the TLE method to produce dose distributions in complex cases and to evaluate the necessary computing time, two simulations were performed using experimental CT images as inputs for defining the geometry and composition of the volume in which the dose deposition would be evaluated. We considered the CT data of an excised and formalin-fixed human knee joint slice ($444 \times 451 \times 1$ voxels, with voxel size $0.3 \text{ mm} \times 0.3 \text{ mm} \times 10 \text{ mm}$) and of a tumour-bearing human breast slice ($321 \times 320 \times 1$ voxels, with voxel size $0.3 \text{ mm} \times 0.3 \text{ mm} \times 10 \text{ mm}$) acquired at 60 keV via high-resolution CT imaging (Snigirev *et al.*, 1995; Förster *et al.*, 1980). The CT images were acquired experimentally using a detector with a pixel size of $50 \mu\text{m}$.

The parallel X-ray beam had a rectangular cross section of $10 \text{ mm} \times 136 \text{ mm}$ in the knee joint measurement and of $10 \text{ mm} \times 97 \text{ mm}$ in the breast measurement. The object was rotated around an axis perpendicular to the beam direction and 200 angular projections over 360° were simulated. The CT volume of the knee joint was segmented using a simple thresholding method in order to attribute each voxel to a specific material (solution of 4% formalin in water, muscle and cortical bone). The breast was segmented into multiple components using a marker-controlled watershed viscous transform (Vachier & Meyer, 2005). By assuming that a three-dimensional image is composed of reliefs, this method is well adapted to the intrinsic nature of the CT images which are characterized by a strong signal on the borders of each feature. The medical *a priori* knowledge of the sample composition was used to assign the following materials to the different regions: adipose tissue, glandular tissue, air and 4% formalin solution in water. In both cases, 10^8 histories were numerically tracked. The simulation parameters were the same as those described in §2.6.3.

In order to test the validity of the local deposition assumption of all energy of the produced secondary electrons, one of the simulations for the breast case was repeated with 10^7 events by activating the tracking of the secondary electrons. The resulting deposited dose is presented in the next section.

3. Results

3.1. Benchmarking against MC simulation

In the test case described in §2.5, we first verified that our TLE implementation in *GATE* leads to the same dose distribution computed by the equivalent MC dose actor available in *GATE*. This is in fact the case within the statistical fluctuations, as shown in Figs. 5 and 6. The integrated deposited energy values calculated with the two methods are in agreement within the given uncertainty (Table 2). The uncertainties are calculated considering the standard deviation

Table 2

Integrated energy with relative uncertainty in the thorax model, computed with the TLE and the MC method, respectively, under the same simulation conditions (4×10^8 incident photons).

MC	TLE	Relative difference
$3.04 \times 10^6 \pm 0.009\% \text{ meV}$	$3.04 \times 10^6 \pm 0.008\% \text{ meV}$	0.064%

of the integrated value of the deposited energy using 20 simulations.

In Fig. 5 and in the profiles in Fig. 6 it can be seen that the uncertainty is much larger in the case of the MC simulation because of the slower convergence, as compared with the TLE method.

3.2. Accuracy test of the TLE method against measurements

The flux values measured with the PTW chamber and the HEGD detector, reported in Table 3, are in satisfactory agreement. From now on, only the values measured with the PTW are taken into account to assess the photon flux necessary for the simulations.

The uncertainties on the HEGD values were calculated using the statistical uncertainties on repeated measurements,

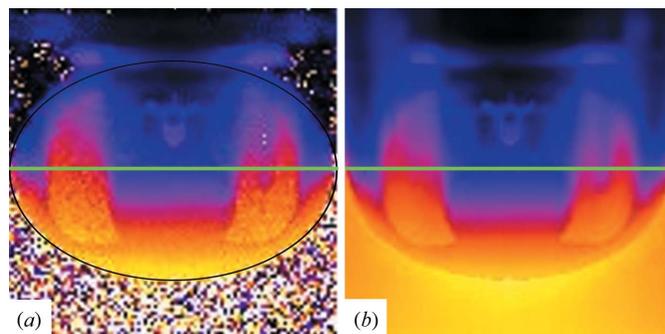


Figure 5

Dose distribution maps obtained in the thorax model with the MC method (a) and the TLE method (b). 4×10^8 incident photons were used. The black oval line in (a) delimits the region considered for the comparison of the total deposited energy shown in Table 2.

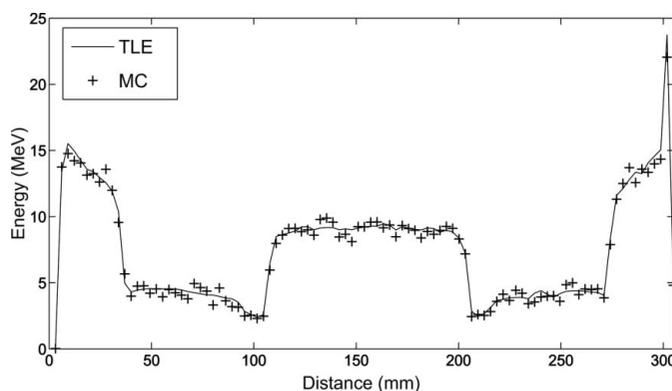


Figure 6

Energy deposition profiles in the thorax model simulated with both TLE and MC methods (4×10^8 incident photons). The profiles correspond to the horizontal section marked by the green line in Fig. 5.

Table 3

Photon flux values obtained from the measurements at different energies with the HEGD and the PTW chamber.

Energy (keV)	HEGD [photons (mm ² mA s) ⁻¹]	PTW [photons (mm ² mA s) ⁻¹]
33.7	$(1.82 \pm 0.14) \times 10^7$	$(1.84 \pm 0.02) \times 10^7$
45.0	$(1.27 \pm 0.06) \times 10^8$	$(1.26 \pm 0.01) \times 10^8$
50.5	$(1.74 \pm 0.09) \times 10^8$	$(1.71 \pm 0.02) \times 10^8$
65.0	$(1.69 \pm 0.09) \times 10^8$	$(1.60 \pm 0.02) \times 10^8$
72.0	$(1.41 \pm 0.07) \times 10^8$	$(1.33 \pm 0.01) \times 10^8$

while for the PTW the uncertainties on both the measurements and the calibration were considered.

In Table 4, the results of the comparison between measured and simulated dose for the CT phantom are reported. The error bars on the simulated values are calculated propagating the uncertainty on the fluence and the statistical error of the simulations. For the measured values, uncertainties are obtained again by propagating the statistical spread of repeated measurements and that of the ionization chamber calibration (this value is provided by the chamber constructor). In Fig. 7, the dose distributions calculated in one transversal slice of the CT phantom are shown, with the CT ionization chamber successively positioned in the three holes. The brighter circular area in the phantom is the volume filled with water (corresponding to the PTW ionization chamber) for which the dose was both measured and simulated.

3.3. Performances comparison in anatomically complex cases

The results of the simulations carried out with the high-resolution CT volumes of the human knee joint and breast are presented in Fig. 8. Relatively smooth dose maps (~0.3% mean statistical error per voxel in the whole knee joint volume and ~0.2% in the breast volume) are obtained using the TLE (with 10⁸ histories) whereas, for the same number of incident photons, equivalent MC simulations give results affected by very high uncertainty. In fact, for the MC case a large number of voxels are left with no dose information at all and this leads to a mean statistical uncertainty per voxel of ~6.8% in the knee joint and ~5.4% in the breast. Contrary to the standard

Table 4

Comparison of simulated and measured doses (in Gy) for the CT geometries of Fig. 7.

The ionization chamber positions refer to Fig. 4. For each position the computations and the measurements were repeated three times because for each measurement the irradiation time of the chamber is slightly different, thus leading to a different number of photons that have to be entered in the simulations.

Position	Measured (Gy)	Simulated (Gy)	% Difference
1	0.262 ± 0.010	0.259 ± 0.007	1.1
1	0.261 ± 0.010	0.259 ± 0.007	0.8
1	0.261 ± 0.010	0.259 ± 0.007	0.8
2	0.264 ± 0.010	0.265 ± 0.007	-0.3
2	0.264 ± 0.010	0.265 ± 0.007	-0.3
2	0.263 ± 0.010	0.265 ± 0.007	-0.7
3	0.261 ± 0.010	0.266 ± 0.007	-1.9
3	0.260 ± 0.010	0.266 ± 0.007	-2.3
3	0.261 ± 0.010	0.266 ± 0.007	-1.9

MC, the use of TLE makes it possible to obtain meaningful dose maps also with a very small number of events (as shown in Fig. 8). For the sake of comparison, integration over all the voxels composing the sample has been performed to work out the average values of the deposited dose. The difference between the average dose values simulated with the standard MC and the TLE method is ~1.2% in the case of the knee image and ~0.6% for the breast test case.

The total computation time was about 11 h for the knee and 9 h for the breast, using ten 3.3 GHz CPUs for the simulations based on the TLE method, whereas, using the MC method, 10 h are required for the knee and 8 h for the breast. These differences in time are due to several additional operations required within the TLE and related to its coefficients database (see §2.3).

It is possible to estimate the number of events, N_f , required to obtain a statistical error of about σ_f , by means of $N_f = (\sigma_i/\sigma_f)^2 N_i$, where σ_i and σ_f are the standard deviations with N_i and N_f events, respectively. It is possible to estimate this number in order to recover the same average statistical error as a TLE calculation for a considered region of interest (*i.e.* the entire volume) (Table 5).

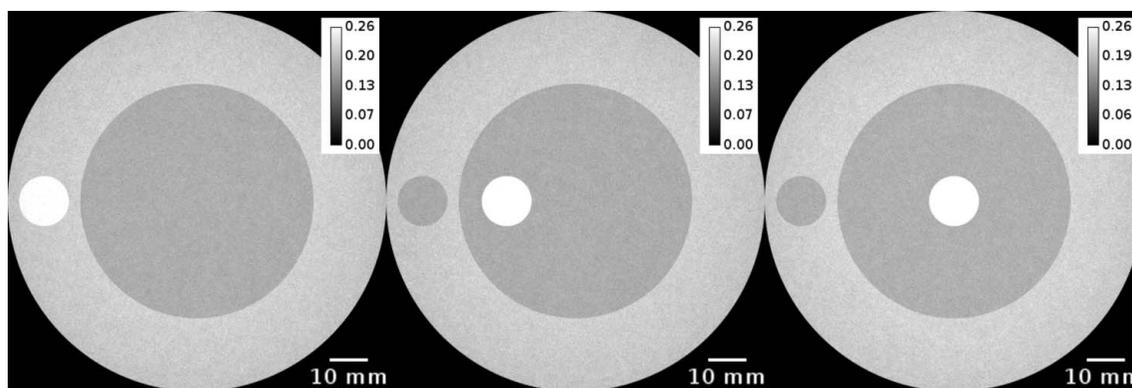


Figure 7

Simulated dose distribution maps of a transversal section of the CT phantom corresponding to three different positions of the ionization chamber (the water-filled volume is the white circular area) as reported in Fig. 4. The grey levels correspond to dose values in Gy. The holes not hosting the ionization chamber are filled with PE.

Table 5
Performance comparison in anatomically complex cases.

Sample	TLE uncertainty	MC uncertainty	N_f/N_i^\dagger
Knee	0.3%	6.8%	~460
Breast	0.2%	5.4%	~650

† The value of N_f/N_i has been corrected considering that the TLE method requires about 10% more computation time with respect to the MC with the same number of histories. This value can be easily related to the gain in speed of the simulation because of the linear correlation between the computation time and the number of histories used in the simulation.

The reason for this very long time is the great number and the small size of voxels in the two presented cases.

In Table 6 the values of the integrated deposited energy calculated with and without forcing the secondary electrons to deposit locally their energy are reported for the breast case scoring 10^7 events: the two estimates are in good agreement within the statistical uncertainty.

4. Concluding remarks

In the present study the implementation of the TLE method in the next release of the open source *GATE/Geant4* 9.5 simulation platform is described. As a result, a new software module named *TLEDoseActor* was created, which uses tables

Table 6
Integrated energy deposition calculated with (asterisk) and without switching the tracking option of the secondary electrons.

Values are expressed in MeV.

TLE*	TLE	Relative difference
$7.98 \times 10^5 \pm 0.015\%$	$7.98 \times 10^5 \pm 0.015\%$	0.033%

of energy-absorption coefficients generated from the EPDL97 database. It is also shown that TLE provides a powerful tool for simulating the dose deposited by photon beams in the keV energy region, when secondary electrons can be deemed to deposit all their energy locally, using a small number of events and without loss of accuracy. Potential applications of this new tool range from CT imaging to nuclear medicine as well as low-energy X-ray external radiotherapy.

The *GATE* implementation of the TLE was benchmarked against analogous MC dose calculations. No significant difference was found between the results of the two methods referring to the total deposited energy. In addition, the spatial distributions of deposited energy were also found to be in agreement within statistical fluctuations. However, the TLE method provides the substantial advantage of exhibiting a strong variance reduction in comparison with equivalent MC simulations, with no significant loss of accuracy. In an anatomo-

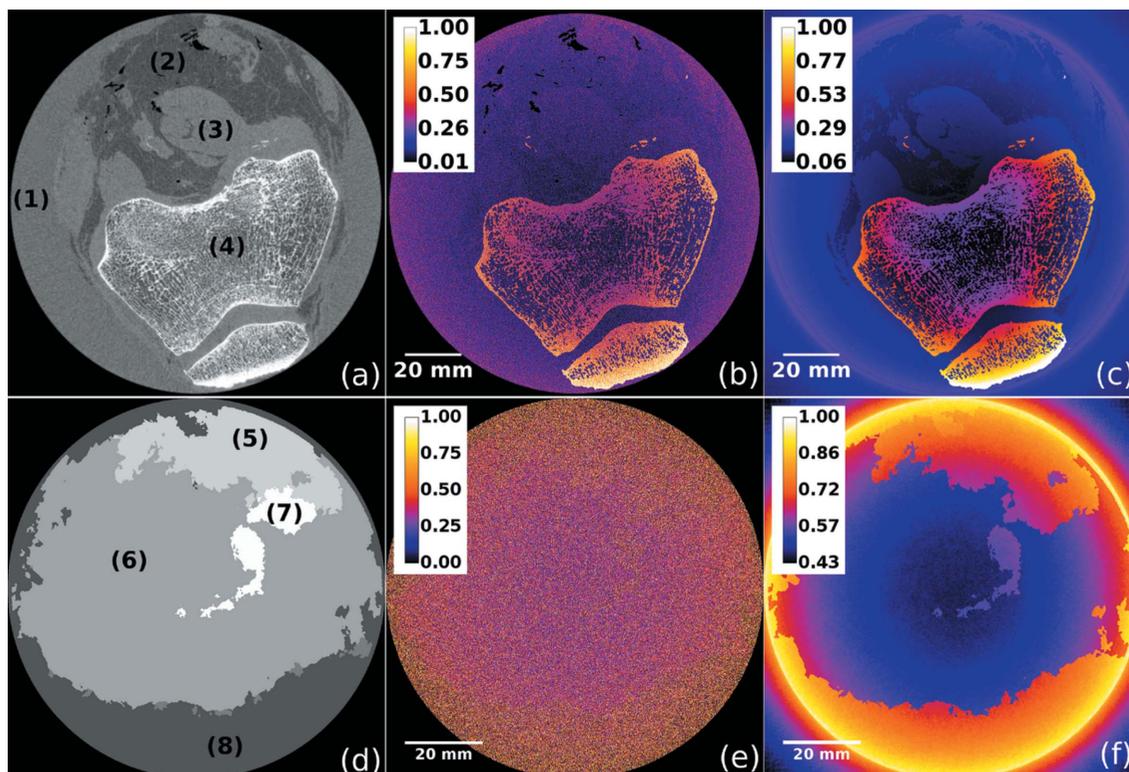


Figure 8
Dose distributions obtained in the human knee joint and breast with the MC (centre) and the TLE method (right), with 10^8 incident photon counts. (a), (b) and (c): knee joint; (d), (e) and (f): breast. (a), (d) are the sample models derived from experimental CT data, (b), (e) are the dose maps obtained with the TLE method, while (c), (f) are the maps computed with the equivalent MC method. In (a), the region marked (1) has been assigned to the formalin solution, (2) to the adipose tissue, (3) to muscle and (4) to the cortical and trabecular bone. In (d), (5) has been assigned to glandular tissue, (6) represents the adipose tissue, (7) is the tumour and (8) is the formalin solution. The dose scales in (b), (c), (e) and (f) are normalized to unity.

mically complex case (*cf.* §3.3) we found that MC computations required about 500 times more events to reach the same averaged statistical uncertainty per voxel as the TLE method, using the same parameters.

The TLE dose calculations were also validated experimentally in the case of a CT dosimetry experiment with a cylindrical phantom irradiated by a 60 keV monochromatic synchrotron beam. Differences between experiment and simulation lower than 2.3% were found.

The applicability of the TLE method is, however, currently limited to an energy range between 1 keV and 1 MeV owing to the limited range of the pre-calculated mass energy-absorption coefficients in the database.

Most importantly, the high-energy limit for applying the TLE approach is fixed by the assumption that the electron transport can be neglected (local energy deposit), with no significant energy escape in the form of secondary radiation, *i.e.* energy-transfer and energy-absorption coefficients are assumed to coincide, this limitation becoming more stringent in the case of high-*Z* elements.

In summary, the TLE method makes it possible to simulate dose distributions in very complex geometries, like organ volumes recorded during high-resolution CT scans (with the identification of the atomic composition of the tissue assigned to each voxel) in several hours, depending on the parameters used, whereas analogous MC simulations requiring equivalent amounts of processing time suffer from much slower statistical convergence. The detailed statistical study of the variance reduction behaviour, which depends on geometric (voxel size), beam configuration and material parameters (density and composition), will be the subject of a future investigation.

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